

# 1 Calculation of Internal Radiation Dose due to Acute Ingestion of 2 $^{60}\text{Co}$ by Adopting HATM Model

3 Haron-Or-Rashid<sup>1</sup>, A.H.M. R. Quddus<sup>2</sup>, M. S. Hossain<sup>3</sup> and K.H. Sarker<sup>4</sup>

4 <sup>1</sup> Bangladesh Army University of Science and Technology (BAUET)

5 *Received: 8 December 2018 Accepted: 4 January 2019 Published: 15 January 2019*

---

## 7 **Abstract**

8 Activity of radionuclide, absorbed dose, committed equivalent dose, committed effective dose  
9 due to acute intake of 1 Bq of  $^{60}\text{Co}$  through ingestion have been calculated by using locally  
10 developed software that has been prepared basing on the human alimentary tract model. Due  
11 to ingestion, maximum radiation dose is deposited in the alimentary tract, which consists of  
12 seven tissue compartments, e.g., OC, OP, ST, SI, LC, RC, and RSC. Tissue masses of  
13 alimentary tract for Bangladeshi people have been considered to calculate the abovementioned  
14 quantities for different age groups such as newborn, 1 yr, 5 yrs, 10 yrs, 15 yrs (male and  
15 female) and adult (male and female). Regarding age the variation of absorbed dose,  
16 committed equivalent dose and committed effective dose follows the sequence: newborn >1  
17 yr > 5 yr > 10 yrs > 15 yrs > adult female > adult male.

---

19 **Index terms**— absorbed dose, committed equivalent, and committed effective dose, human alimentary tract  
20 model (HATM)  
21 radioisotope production laboratory and research facility or during routine work at the workplaces with an  
22 unsealed radioisotope. That's why the authorities such as UNSCEAR [2], IAEA [3], and ICRP [4] develop  
23 radiation safety standards. Internal radiation dose cannot be measured directly; of course, some models are there.  
24 This can be used for assessment of internal radiation dose, based on the radioactivity by bioassay measurement  
25 and whole-body counting.

26 The present study describes a generic methodology for the calculation of internal radiation doses due to the  
27 acute intake of beta-emitting radionuclides through ingestion. Visual Basic language software has been developed.  
28 The software is userfriendly and is found to work well as desired. This software can comfortably be used for  
29 calculation of internal radiation doses due to the intake of radioisotopes through ingestion by radiation workers  
30 and the public at large.

31 The activity of radionuclide, absorbed dose, committed equivalent dose, committed effective dose due to acute  
32 intake of 1 Bq of  $^{60}\text{Co}$  through ingestion have been calculated by using the software that has been prepared  
33 based on the HATM. Due to ingestion maximum radiation dose is deposited in the alimentary tract, which  
34 consists of seven tissue compartments, (RC) and Rectosigmoid Colon (RSC). Tissue masses of alimentary tract  
35 for Bangladeshi people have been considered to calculate the above-mentioned quantities for different age groups  
36 such as newborn, 1 yr, 5 yrs, 10 yrs, 15 yrs (male and female) and adult (male and female).

## 37 **1 a) The HATM**

38 There are various ICRP, and MIRD models that are similar in terms of their assumption and defining equation.  
39 Contemporary internal dosimetry models began with the single compartment models of ICRP [5], and [6]. The  
40 MIRD methodology [7][8][9] and ICRP [10] and [11] developed the concept of source and target organs. ICRP  
41 [12] and ICRP [13] continue to refine to internal dosimetry model.

42 The new human alimentary tract model (HATM) [14] considers the movement of radionuclides Introduction  
43 adionuclides once entered into the body through different routes of entry [1] can't be eliminated. It gives out

## 4 KG PER TRANSFORMATION

44 energy continuously as long as it remains inside the body. So it is necessary to assess internal radiation dose to  
45 measure the risk of human health. Occupational workers, and the public can be internally exposed by radiation  
46 due to the ingestion of contaminated food following nuclear reactor accident, accidental intake during the use of  
47 unsealed radioisotope in the field of nuclear medicine, throughout the tract from ingestion to elimination. The  
48 model (HATM) includes compartments representing the oral cavity (OC) and esophagus (OP) to account for  
49 doses received from transit or retention of activity in the upper regions of the tract. The model partitions the  
50 large intestine into three parts frequently addressed in colonic transit studies. It also includes compartments to  
51 account for nuclear transformations due to retention of a radionuclide in tissues of the tract in those cases where  
52 tissue retention is found by available information. The model includes pathways to account for absorption from  
53 the oral mucosa, stomach, or segments of the colon if specific information is available. HATM provides age and  
54 gender-specific transit times for all of the tract depicted in the model and, for the upper (oral cavity, esophagus,  
55 and stomach), also provides materials-specific transit times.

56 The 1990 recommendations of ICRP introduced specific risk estimates and tissue weighting factors, w T , for  
57 radiation-induced cancer of the esophagus, stomach, and colon, requiring dose estimates for each of these regions.  
58 HATM takes account of sites of radionuclide absorption and retention in the tract and routes of excretion of  
59 absorbed radionuclides into the tract. Doses are calculated for sensitive cells in each region: mouth, esophagus,  
60 stomach, small intestine, and colon.

61 2 II.

### 62 3 Methodology

96 Where, A(t) is the activity at any organ after a time t from ingestion. T R i i i M w S T AF E Y S T SEE )  
97 ( ) ( ? = ? MeV 1 ?

## 98 4 Kg per transformation

99 Where,  $Y_i$  is the yield of radiations of type  $i$  per transformation  $E$   $i$  is the average, or unique energy of radiation  
 100  $i$  in MeV  $AF(T) ? S$  is the absorbed fraction that is the average fraction of energy in  $T$  from radiation arising in  
 101  $S$ ;  $WR$ , the radiation weighting factor and  $M_T$  is the mass of the target organ in kg.

---

## 102 5 c) Committed Equivalent Dose

103 The committed equivalent dose for each type of radiation is given by  $s_i S T \text{ SEE } U S T H$  ( 10 6 . 1 ) ( 13 ?  
104  $\times \times \times = ? ? \text{ Sv}$  (11)

105 Where  $s_i$  is the number of the transformation of  $j$  in  $S$  over the lifetime following intake of the radionuclide.  
106 This is the expression for the number of transformations in the various organs in the tract following ingestion of  
107 1 Bq of activity.

108 Oral cavity:  $R \text{ OC OC } U ? ? + = 1$  Esophagus:  $) ( ( 1 R EP R OC EP U ? ? ? ? + + = \text{Stomach: } ) ) ( ($   
109  $1 R ST R EP R OC ST U ? ? ? ? + + + = \text{Small intestine: } ) ) ( ( ( 1 B R SI R ST R EP R OC ST U ?$   
110  $? ? ? ? ? ? + + + + = \text{Left colon: } ) ) ( ( ( ( 1 R LC B R SI R ST R EP R OC LC U ? ? ? ? ? ?$   
111  $? ? ? ? + + + + + = \text{Right colon: } ) ) ( ( ( ( ( 1 R RC R LC B R SI R ST R EP R OC RC U ? ? ? ? ?$   
112  $? ? ? ? ? ? + + + + + + = \text{Rectosigmoid colon: } ) ) ( ( ( ( ( ( 1 R RSC R RC R LC B R SI R ST$   
113  $R EP R OC RSC U ? ? ? ? ? ? + + + + + + = d)$  Committed Effective Dose

114 Committed effective dose for any organ of alimentary tract is the product of committed equivalent dose and  
115 tissue weighting factor  $i W_i H E \times ? = )$  ( ? mSv (12)

116 Where  $W_i$  is the tissue weighting factor.

## 117 6 Global Journal of Medical Research

118 Volume XIX Issue II Version I

## 119 7 Results and Discussion

120 Activity, absorbed dose, committed equivalent dose, and committed effective doses due to acute ingestion of 1  
121 Bq of 60 Co. Tissue masses of alimentary tract for Bangladeshi people have been considered to calculate the  
122 above-mentioned quantities for different age groups such as newborn, 1 yr, 5 yrs, 10 yrs, 15 yrs (male and female)  
123 and adult (male and female).

## 124 8 a) Activity

125 Activity has been calculated at different compartments of HAT of the subjects of age groups: newborn, 1 yr, 5  
126 yrs, 10 yrs, 15 yrs (male), 15 yrs (female) and adult (both male and female) and time elapsed as considered in the  
127 work is mostly 0.5 hr, 1 hr, 2 hrs, 4 hrs, 8 hrs, 12 hrs, 24 hrs and 48 hrs after the ingestion of the radionuclide.

128 Figs. ??-7 show the variation of activity in OC, OP, ST, SI, LC, RC, and RSC for all age groups. By studying  
129 the nature of the graphs, it is found that the tissues of all the seven organs (excepting oral cavity) show a tendency  
130 of rising of activity initially and subsequent falling. The radionuclide 60 Co is absorbed in one organ, which is  
131 caused after the release of it from the previous organ. The significant aspects of the absorption in and release  
132 from these organs are described below:

133 The activity-time graph for 60 Co has been constructed for the above-mentioned seven tissues of the alimentary  
134 tract. By studying the nature of the graphs it is found that the tissues of all the seven organs (excepting oral  
135 cavity) show a tendency of the rise of activity initially and subsequent falling.

136 Activity-versus-time graphs are plotted for OC (Fig. ??), OP (Fig. ??), and ST (Fig. ??). Fig. ?? shows  
137 that for OC, at around 0.001 hr after ingestion of the radionuclide, the activity reaches 0.97 Bq, and at 0.2hr  
138 after ingestion, it reaches to around  $2.47 \times 10^{-3}$  Bq. The observation (a sharp fall) can be accepted because OC  
139 is the first organ, and its transit time is very short.

140 The excreted radionuclide will then appear in the next tissue, e.g., OP. After the lapse of the time at OC, the  
141 activity in OP should show growth, and this is observed in work; the peak is found to appear at around 0.02 hrs  
142 after ingestion. The value in the OP attains the maximum value at this time, the calculated result and the rising  
143 rate being 0.19Bq and 9.5Bq/hr, respectively. The activity change with time shows a sharp fall. After 0.4hrs,  
144 the activity level reduces to  $3.07 \times 10^{-6}$  Bq. This time is also short, again possibly due to the low transit time of  
145 the organ.

146 The excreted radionuclide is then deposited in the later tissue, e.g., ST. Fig. ?? shows that for ST, the activity  
147 level reaches to the maximum value ( $=0.89$ Bq) at 0.15hrs after ingestion. The rising-rate is 5.9Bq/hr. Then the  
148 activity level decreases exponentially. Finally, it reduces to a value of approximately  $1.97 \times 10^{-4}$  Bq at a time  
149 10hrs after ingestion.

150 The excreted radionuclide from ST is then deposited in the later tissue, e.g., SI. Fig. ?? shows the pattern of  
151 change. In the case of SI, the maximum value of activity appears at about 1.2hrs after the ingestion; the maximum  
152 significance being 0.36 Bq. Then it falls, and in doing so, it takes a time of about 15hrs in total to reach the  
153 value of  $2.29 \times 10^{-5}$  Bq.

154 The radionuclide then goes to the next tissue, e.g., LC, and the pattern of retention in the organ is shown in  
155 Fig. ?? In LC, activity level rises to 5hrs after ingestion, which is remarkably different from that of the other  
156 organs. The maximum value attained is 0.73 Bq. During the falling down process, this organ takes a very long  
157 time, e.g., approximately 96 hrs to reach it of approximately  $4.2 \times 10^{-4}$  Bq.

158 Fig. ?? shows the variation of the activity with time for the organ RC. One may observe from the figure  
159 that in the case of RC, it rises up to attain the maximum value ( $=0.36$ ) in 15hrs of duration; the rising rate  
160 being 0.024Bq approximately. The activity value then continuously decreases, and after an elapse of 150 hrs, the

## 12 CONCLUSION

---

161 organ retains approximately  $6.31 \times 10^{-5}$  Bq of activity in total. The falling rate is guided by an approximately  
162 exponential function.

163 The RSC graph (Fig. ??) shows that up to around 27 hrs after ingestion, the activity rises, being significantly  
164 different from that of the other ones. The peak value is about 0.27Bq, the rising rate being 0.01Bq/hr. Then the  
165 activity level decreases exponentially. Around 180hrs, duration is necessary for the activity level to fall to the  
166 value of  $5.09 \times 10^{-5}$  Bq.

### 167 9 b) Absorbed Dose

168 Figs. 8-13 show the variation of absorbed dose in 0 to 48 hrs by OP, ST, SI, LC and RC organs for a new-born  
169 baby who is supposed to have ingested 1 Bq of the radionuclide 60 Co. The absorbed dose in OP decreases very  
170 rapidly, its value becoming practically insignificant after around 0.4 hours. The absorbed dose in OP, ST, SI, LC,  
171 RC, and RSC increases exponentially (approximately) and then decreases. This pattern of variation is expected  
172 mainly because of the biological excretion phenomenon. Of course, the effect of radioactive half-life is also active  
173 in these cases.

174 The maximum absorbed dose per Bq intake of 60

175 Co is found to be  $1.4 \times 10^{-11}$ ,  $1.95 \times 10^{-11}$ ,  $1.84 \times 10^{-12}$ ,  $1.60 \times 10^{-11}$ ,  $7.89 \times 10^{-12}$ , and  $1.38 \times 10^{-11}$  mSv in  
176 the compartments of OP, ST, SI, LC, RC, and RSC respectively. The maximum absorbed dose in OP occurs  
177 quickly after the process of intake. Similar results are found for all the other age groups: 1 yr, 5 yrs, 10 yrs, 15  
178 yrs (male), 15 yrs (female), adult (male) and adult (female). The absorbed dose in different parts of the human  
179 alimentary tract for the adult (male) is found to be the lowest because of the relatively larger tissue

### 180 10 c) Committed Equivalent Dose

181 Figs. [14][15][16] show the variation of committed equivalent dose in OP, LC, ST, SI, RC, and RSC for eight  
182 different age groups of people due to ingestion of the radionuclide 60 Co. The equivalent dose is the maximum in  
183 the case of subjects of new-born age group. Then it decreases as age increases; its value is becoming almost the  
184 same for 15 yrs (male), 15 yrs (female), adult (male), and adult (female) because of their having approximately  
185 similar body mass [16].

186 The maximum committed equivalent dose per Bq intake of 60 Co is found to be  $3.07 \times 10^{-9}$ ,  $9.21 \times 10^{-8}$ ,  
187  $2.08 \times 10^{-8}$ ,  $2.59 \times 10^{-7}$ ,  $2.59 \times 10^{-7}$ , and  $6.05 \times 10^{-7}$  mSv for OP, ST, SI, LC, RC, and RSC respectively. Fig.  
188 ??7 shows the variation of committed equivalent dose in OP, LC, ST, SI, RC, and RSC for a particular age group  
189 of subjects, e.g., newborn child. The committed equivalent dose has a minimum value in OP due to a very tiny  
190 number of transformations (only 40) occurring there. In the next organ, e.g., ST, this value rises due to its larger  
191 number of transformations. In SI, this value is again decreasing due to its larger mass. In LC and RC, this value  
192 is almost the same because of their equal mass and transformation number. In RSC committed equivalent dose  
193 is maximum due to its lowest mass. For age groups: 1 yr, 5 yrs, 10 yrs, 15 yrs (male), 15yrs (female), adult  
194 (male), and adult (female) similar results are found.

195 The variation of committed effective dose in the organs OP, ST, SI, LC, RC, and RSC for the different age  
196 groups of people due to the ingestion of the radionuclide 60 Co is shown in Figs. 18-20. As expected, the  
197 committed effective dose is the maximum in case of a subject of new-born age group. Then it decreases as  
198 age increases; its value is becoming almost the same for 15 yrs (male), 15 yrs (female), adult (male) and adult  
199 (female) subjects because of their having close body mass.

200 The maximum committed effective dose per gram intake of 60 Co is found to be  $1.22 \times 10^{-10}$ ,  $1.10 \times 10^{-8}$ ,  
201  $2.41 \times 10^{-9}$ ,  $3.11 \times 10^{-8}$ ,  $3.11 \times 10^{-8}$ , and  $7.26 \times 10^{-8}$  mSv for OP, ST, SI, LC, RC, and RSC respectively. In the  
202 case of a new-born baby, the variation of committed effective dose in the organs OP, ST, SI, LC, RC, and RSC  
203 is given in Fig. ??1.

204 Committed Effective dose has a minimum value in OP due to a very tiny number of transformations. In the  
205 next organ e.g., ST this value is rising due to its greater number of transformations. In SI, this value is again  
206 decreasing due to its larger mass. In LC and RC this value is almost the same because of their equal mass and  
207 transformation number. In RSC, committed equivalent dose is the maximum due its lowest mass.

208 Similar results are found for subjects of age groups: 1 yr, 5 yrs, 10 yrs, 15 yrs (male), 15yrs (female), adult  
209 (male), and adult (female).

## 210 11 IV.

### 211 12 Conclusion

212 Due to ingestion, maximum radiation dose is deposited in the alimentary tract, which consists of seven tissue  
213 compartments, e.g., OC, OP, ST, SI, LC, RC, and RSC. The transfer of radionuclides from the oral cavity to the  
214 esophagus has been considered an instantaneous process that gives less retention but activity in the entry route.

---

Regarding age, the variation of absorbed dose, committed equivalent dose, and committed effective dose follows the sequence: Newborn >1 yr > 5 yr > 10 yrs > 15 yrs > adult female > adult male.

- c. Absorbed dose for an alpha-emitting radionuclide is higher than beta-emitting radionuclides due to higher radiation weighting factor ( $w_R$ ).
- d. The absorbed dose, committed equivalent, and committed effective dose show a common tendency that these values are maximum for a subject of newly born age group; then, it decreases as the age increases for all the radionuclides of interest.
- e. Regarding compartment the trends of variation of maximum absorbed dose are: ST > LC > OP > RSC > RC > SI
- f. Regarding tissue compartments the variation pattern of committed equivalent dose is: RSC > LC > RC > ST > SI > OP
- g. The highest committed effective dose per Bq intake for each radionuclide is found in the alimentary tract of a newborn baby. These values for stomach are  $3.72 \times 10^{-6}$  mSv/Bq,  $2.16 \times 10^{-6}$  mSv/Bq,  $8.64 \times 10^{-6}$  mSv/Bq, and  $1.12 \times 10^{-6}$  mSv/Bq for  $^{239}\text{Pu}$ ,  $^{233}\text{U}$ ,  $^{232}\text{Th}$ , and  $^{226}\text{Ra}$  respectively.

Figure 1:

## 12 CONCLUSION

---

215 The following important observations could be made from the study: a. <sup>1 2 3</sup>

---

<sup>1</sup>© 2019 Global Journals Calculation of Internal Radiation Dose due to Acute Ingestion of 60 Co by Adopting HATM Model

<sup>2</sup>© 2019 Global Journals

<sup>3</sup>D © 2019 Global Journals Calculation of Internal Radiation Dose due to Acute Ingestion of 60 Co by Adopting HATM Model

---

216 [Jones ()] 'A realistic anthropomorphic phantom for calculating specific absorbed fractions of energy deposited  
217 from internal gamma emitters'. D G Jones . *Radiat. Prot. Dosim* 1998. 79 p. 411.

218 [Snyder et al. ()] *Absorbed Dose per Unit Cumulated Activity for Selected Radionuclides and Organs, MIRD  
219 Pamphlet No*, W Snyder , M Ford , G Warner , S Watson . 1975. New York. (Society of Nuclear Medicine)

220 [Age-Dependent Doses to Members of the Public from Intake of Radionuclides: Part 5, ICRP Publication 72 International Comm  
221 'Age-Dependent Doses to Members of the Public from Intake of Radionuclides: Part 5, ICRP Publication  
222 72'. *International Commission on Radiological Protection* 1996. Elsevier Science. 26. (Ann. ICRP)

223 [Quddus ()] *Assessment of Internal Radiation Doses due to Intake of Radionuclides by Ingestion in Human Body*,  
224 A H M Quddus . 2010. Savar, Bangladesh. Physics Department Jahangirnagar University (PhD Thesis)

225 [Basic Anatomical and Physiological Data for use in Radiological Protection Reference Values ICRP Publication]  
226 'Basic Anatomical and Physiological Data for use in Radiological Protection Reference Values'. *ICRP  
227 Publication* 89. (Annals of ICRP)

228 [Effects of Ionizing Radiation, United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR) ()]  
229 *Effects of Ionizing Radiation, United Nations Scientific Committee on the Effects of Atomic Radiation  
230 (UNSCEAR)*, 2000. UNSCEAR. (Report)

231 [Evaluation of Radiation Doses to Body Tissues from Internal Contamination due to Occupational Exposure]  
232 *Evaluation of Radiation Doses to Body Tissues from Internal Contamination due to Occupational Exposure*,  
233 ICRP Publication.

234 [Human Alimentary Tract Model for Dose Calculation ICRP Publication] 'Human Alimentary Tract Model for  
235 Dose Calculation'. *ICRP Publication* Pergamon Press. 100.

236 [Human Respiratory Tract Model for Radiological Protection ICRP ()] 'Human Respiratory Tract Model for  
237 Radiological Protection'. *ICRP* 1994. Pergamon Press, ICRP Publication 66.

238 [Iaea and Series ()] 'International Basic Safety Standards for Protection against Ionizing Radiation and for the  
239 Safety of Radiation Sources'. Safety Iaea , Series . *International Atomic Energy Agency* 1996. (115) .

240 [Ionizing Radiation-Safety Standards for the General Public ICRP ()] 'Ionizing Radiation-Safety Standards for  
241 the General Public'. *ICRP* 1991.

242 [Limits for Intakes of Radionuclides by Workers, ICRP Publication 30, Part 1 ICRP ()] 'Limits for Intakes of  
243 Radionuclides by Workers, ICRP Publication 30, Part 1'. *ICRP* 1979. Pergamon Press.

244 [Limits for Intakes of Radionuclides by Workers, ICRP Publication 30, Part 1 ICRP ()] 'Limits for Intakes of  
245 Radionuclides by Workers, ICRP Publication 30, Part 1'. *ICRP* 1979. Pergamon Press.

246 [Loevinger et al. ()] 'MIRD Primer for Absorbed Dose Calculations'. R Loevinger , T Budinger , E Watson .  
247 *Society of Nuclear Medicine* 1988.

248 [Permissible Dose for Internal Radiation] *Permissible Dose for Internal Radiation*, ICRP Publication.

249 [Recommendation of the International Commission on Radiological Protection (ICRP Publication 60 Annals of ICRP ())]  
250 'Recommendation of the International Commission on Radiological Protection (ICRP Publication 60'. *Annals  
251 of ICRP* 1990. 1994. Pergamon press. 21. (ICRP (International Commission on Radiological Protection))